Stand Up of Uranium Capability for Swipe Analysis

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The INL is a U.S. Department of Energy National Laboratory operated by Battelle Energy Alliance

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Prepared for the
U.S. Department of Defense
and for the
U.S. Department of Energy
Under DOE Idaho Operations Office
Contract DE-AC07-05ID14517

Introduction

The INL has established the capability to process and analyze swipe samples to determine if the amount of U and Pu present on equipment and facilities are at the level typical for natural background, to quantify their isotopic composition and to determine if any off-normal isotopic ratio present in the sample is statistically relevant. A previous report detailed this capability for Pu and preliminarily for U; this report describes the measurements and analysis that were performed to demonstrate the INL capability for U.

To establish that a piece of equipment is not contaminated with the element to be sampled, a fabric swipe is used to collect a sample of the materials present on the surface. The swipes are then processed and analyzed to determine if Pu and U are present on the sample at levels above what is accepted as natural background and, for the case of U, whether the isotope ratios deviate from the accepted natural background levels. Both the method applied for chemical processing of the swipes to remove and isolate the U and Pu and the method used to analyze the extracts influences the sensitivity and specificity. Over the years various methods have been developed for processing and analyzing these types of samples; the gold standard for these measurements involves a lengthy and complex separation process followed by analysis using thermal ionization mass spectrometry (TIMS). However, this method is expensive and time consuming, thus driving a need for a less complicated and more efficient method that provides the necessary level of sensitivity and specificity.

Advances in Inductively Coupled Plasma Mass Spectrometry (ICPMS) over the last decade have enabled analyses of U and Pu that rival that of TIMS. This, coupled with the potential for simplifying the extraction and separation process required for an ICPMS analysis, prompted the INL's development of methods that provide the analysis of swipes in a timely and efficient manner.

U is present in the blank swipe material at nanogram (~2 x 10⁻⁹ g) levels for a typical sample, a level easily detected with ICPMS. The abundance of the isotopes ranges over 4 orders of magnitude for the naturally occurring ²³⁴U, ²³⁵U and ²³⁸U and a goal was set to be able to detect the presence of ²³⁶U at 6 orders of magnitude lower than the ²³⁸U. The ²³⁶U measurement is particularly important because the presence of ²³⁶U is a strong indicator that the uranium as been in a nuclear reactor. To demonstrate these capabilities the following sample types were used: blank swipe material, blank process reagents, swipe material spiked with a natural abundance U isotope standard, swipe material spiked with an environmental standard (Columbia River sediment), and swipes taken at various locations within the processing laboratories and the INL environment.

This report summarizes the method used to extract the U from the swipe material, the ICPMS analyses that demonstrate the limit of detection (LOD) and the limit of quantification (LOQ) for the U isotopes of interest, the precision of the measured isotope ratios and the dependence of precision on the quantity of U present, and the method proposed to determine if an off-normal ratio is statistically relevant.

Sample Preparation and Processing

The methods discussed in this report were developed to analyze environmental samples collected on a commercial swipe material. As uranium is ubiquitous in the environment it is typically found in the materials and reagents used to collect and process the samples. Thus any measurement of uranium must account for the possible influence of this background on quantification and on the various isotope ratios. For these studies background levels of uranium and the associated ratios were measured for samples of the blank swipe material and the reagents.

To measure the quantity (mass) of each of the target isotopes in the samples a spike of ²³³U (CRM 111-A, New Brunswick Laboratory) was added to each sample before processing. Serial dilutions of the standard were performed and tracked by weight to produce the concentration needed for spiking individual samples with 3.3 pg of ²³³U.

Three categories of samples were produced and analyzed: swipe material with a natural uranium standard (CRM 145, New Brunswick Laboratory, see Appendix A) applied prior to processing, swipe material with a sediment standard (CRM 4350B, NIST) with a non-natural uranium isotopic composition applied prior to processing, and samples generated by swiping a variety of surfaces within the processing laboratory and the general INL environment.

Sample processing took place in clean rooms located in the IRC-611 building located at the INL Research Center (IRC) in Idaho Falls. Sample analysis was conducted in a standard laboratory located in the IRC-603 building which is ~ 500 yards from IRC-611. The samples were transferred from the clean room to the analysis laboratory in sealed vials.

The samples were initially dry-ashed in a muffle furnace followed by a wet-ash performed in a Teflon beaker using nitric, hydrofluoric and perchloric acids. Chemistry methods, developed by Eichrom, using commercially available UTEVA and TRU columns were used to separate the Pu and U fractions. The sample was passed through a UTEVA column which has an affinity for uranium but not plutonium (the TRU column was staged below the UTEVA column to collect the Pu fraction). The UTEVA cartridge was rinsed and the uranium eluted from the column to produce the samples for the ICPMS.

ICPMS Analysis Method and Data Processing

A Neptune Plus ICPMS was used for all sample analysis. The instrument is configured with both Faraday cups and ion counting detectors that enable simultaneous detection of the U isotopes of interest; it has been described in detail elsewhere. The samples were diluted to a level where the ²³⁵U and ²³⁸U were detected with a faraday cup and the ²³³U, and ²³⁶U masses were detected with ion counting detectors. A cross calibration of each ion counter to the faraday cup used to measure the ²³⁸U was performed at the beginning of each day. Dead time corrections and dark noise corrections were quantified and the instrument was tuned for optimum response each day. A blank was run every other sample and standards were injected to bracket every three samples. A blank swipe

that was processed with the sample set was run at the beginning of a sample sequence and at the end. All data was collected and processed with the Neptune Plus software.

The data was corrected for detector cross calibration and blank subtracted. The blank subtraction was performed by using the average of the blank injection just before and just after each sample run. Mass bias was corrected using the CRM145 standard injected to bracket every three samples. The following equations were used to calculate the mass bias correction:

$$K_{A/B} = \left(\frac{massA}{massB}\right)^{\beta}$$

 $K_{A/B}$ = mass bias correction for the isotopes of mass A and mass B $massA = ^{234}$ U as an example $massB = ^{238}$ U as an example $\beta = defined$ by the equation 2 below

Equation 2

$$\beta = \frac{\ln\left(\left(\frac{A}{B}\right)_{std} / \left(\frac{A}{B}\right)_{meas}\right)}{\ln\left(\frac{massA}{massB}\right)}$$

$$(A/B)_{std} = (^{234}\text{U}/^{238}\text{U})_{std} = \text{certificate value of the }^{234}\text{U}/^{238}\text{U ratio as an example.}$$

 $(A/B)_{meas} = (^{234}\text{U}/^{238}\text{U})_{meas} = \text{measured value of the }^{234}\text{U}/^{238}\text{U ratio as an example.}$

 β is calculated using the average of the 234 U / 238 U ratios of the bracketing CRM 145 standards, and substituted in the first equation to calculate the mass bias correction for each of the other ratios.

A goal in the development of these methods was to be able to detect ²³⁶U at a level 6 orders of magnitude below that of the ²³⁸U. Two corrections were needed to determine the proportion of the signal at mass 236 that is attributable to ²³⁶U: correction for ²³⁵U¹H and correction for baseline tailing of the 235 and 238 signals. The hydride ratio was estimated using the ratio of the 239 mass signal to the 238 mass signal of a blank processed swipe run on the ICPMS at the beginning and the end of the sample set. This makes the assumption that the 239 mass signal did not include any contribution from ²³⁹Pu, an assumption verified by noting that signals of other Pu isotopes were not detected. The estimated hydride contribution from the ²³⁵U¹H was subtracted from the gross 236 signal. The baseline correction to account for tailing from the mass 238 and mass 235 signals into the mass 236 signal was calculated using the average of the signals at mass 235.5 and 236.5. Mass 233, 234, 235 and 238 signals are used as measured with no hydride or tailing corrections.

Results

The measurements from the standards, blank swipes, the process reagents, and the three categories of samples are reported in this section.

CRM 145 Standard

Figures 1 and 2 present the measured isotope ratios of the CRM 145 standard. The results demonstrate that the method produces an accurate measurement of both the $^{235}\text{U}/^{238}\text{U}$ and $^{234}\text{U}/^{238}\text{U}$ ratio. The 2 sigma uncertainty is $\sim 0.4\%$ for the $^{235}\text{U}/^{238}\text{U}$ ratio and 6% for the $^{234}\text{U}/^{238}\text{U}$ ratio. The measured limit of detection for ^{238}U from the CRM 145 spiked onto a swipe is 7.1 x 10^{11} atoms of ^{238}U (0.28 ng) with the limit of quantification being 2.4 x 10^{12} atoms of ^{238}U (0.93 ng). The uncertainty for all of the measurements is a function of the mass of uranium in the sample. Figure 3 illustrates the dependence of the 1 sigma uncertainty on the mass of uranium in the sample.

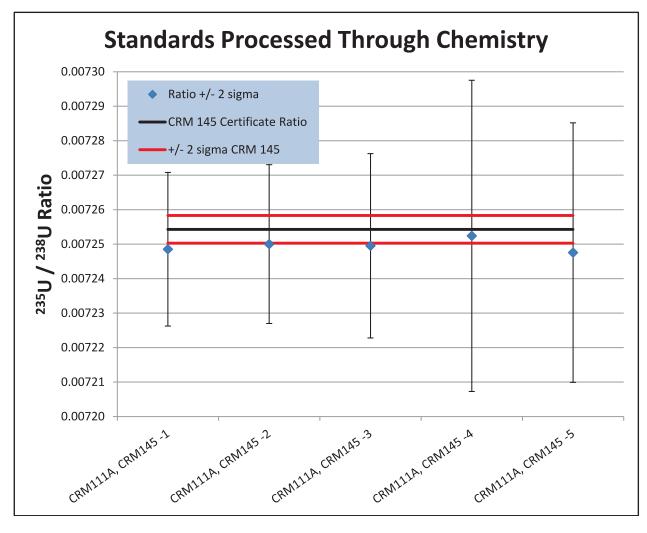


Figure 1. Measured 235 U / 238 U ratios of standards CRM 111A (3.3 fg) and NBS CRM 145 (22 ng) processed through the chemistry. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

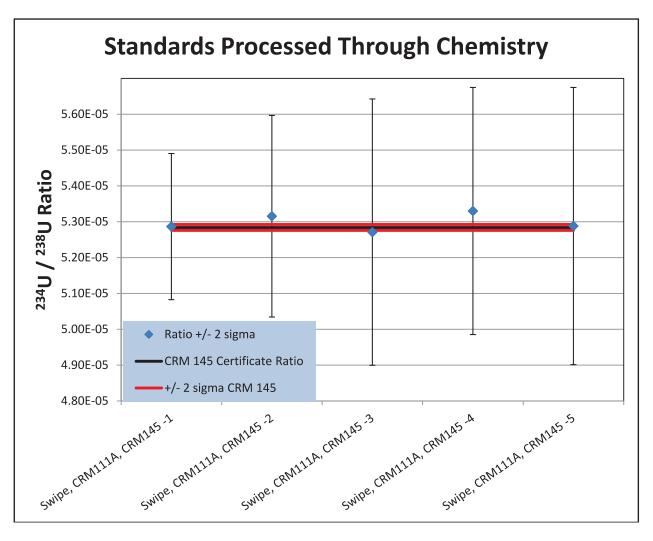


Figure 2. Measured 234 U / 238 U ratios of standards CRM 111A (3.3 pg) and NBS CRM 145 (22 ng) processed through the chemistry. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

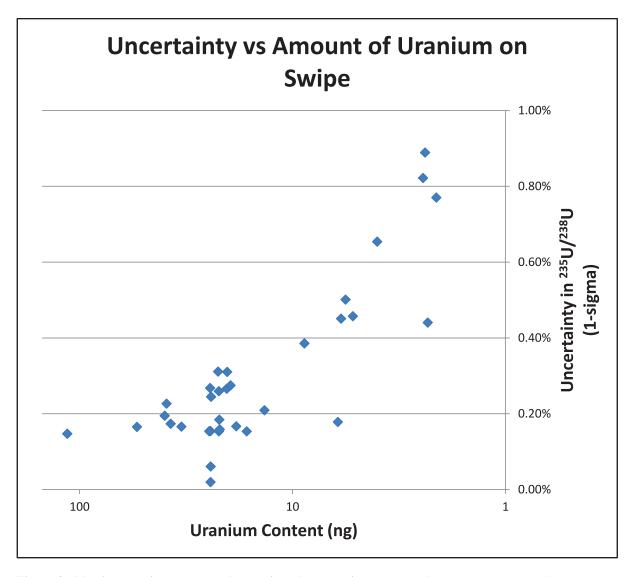


Figure 3. Plotting uranium mass on the x-axis and uncertainty expressed as a percentage on the y-axis. Larger quantities of uranium lead to smaller uncertainty. As the quantity of uranium decreases, the magnitude of the uncertainty increases.

Blank Swipes

Figures 4 and 5 present the measured isotope ratios for ²³⁵U/²³⁸U and ²³⁴U/²³⁸U from blank sample swipes. The measured ²³⁵U/²³⁸U ratio, within the uncertainty bands, corresponds to natural abundance. The ²³⁴U/²³⁸U ratio is enriched in ²³⁴U relative to natural; it is not uncommon to see this level of variation in uranium isotope ratios due to the inhomogeneous distribution of uranium in the environment. It would also not be unreasonable to expect that this ratio may differ from batch to batch of the swipe material; thus it should be quantified for each distinct batch. The amount of uranium in a blank swipe is about 2.5 ng, which for a typical swipe of an environmental sample would represent only about 10% of the total sample. ²³⁶U was not detected in these samples.

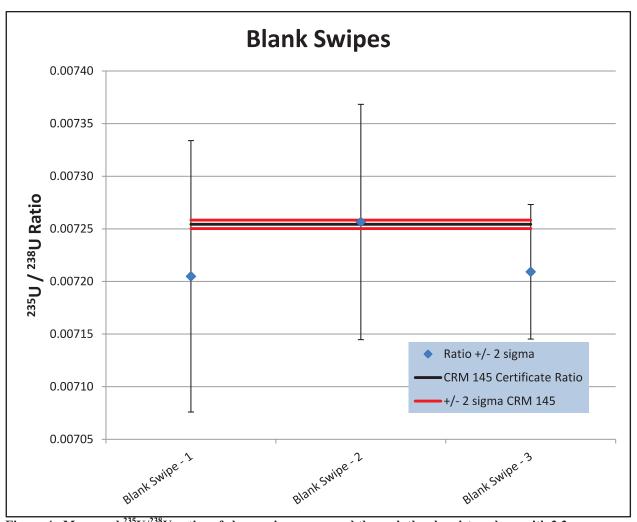


Figure 4. Measured ²³⁵U/²³⁸U ratios of clean swipes processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

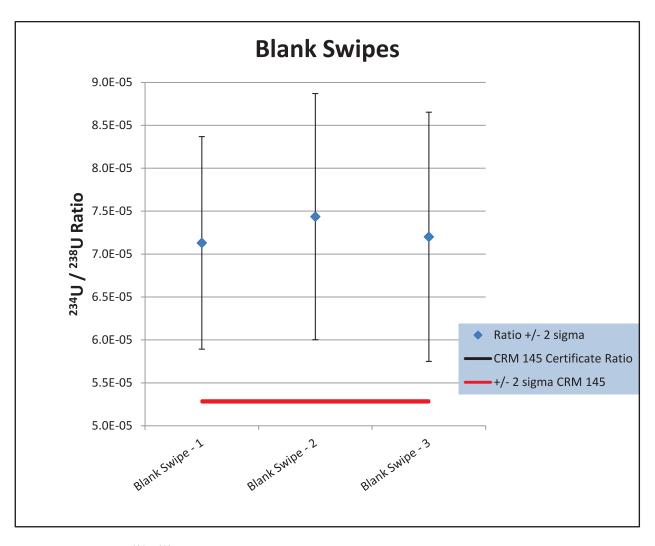


Figure 5. Measured 234 U/ 238 U ratios of clean swipes processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Blank Reagents

Figures 6 and 7 present the measured isotope ratios for 235 U/ 238 U and 234 U/ 238 U from the reagents used in the process. The 235 U/ 238 U ratio, within the uncertainty bands, corresponds to natural abundance. The 234 U/ 238 U may be slightly enriched in 234 U; however, the large uncertainty band, due to the low level of U in the sample, overlaps the normal ratio. 236 U was not detected in these samples.

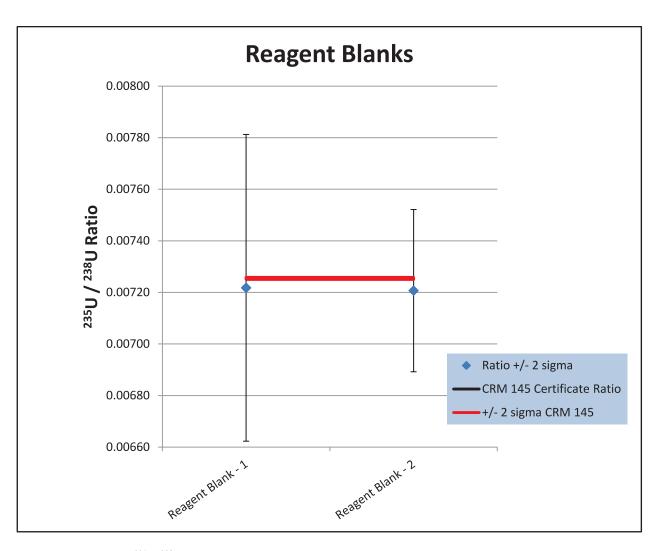


Figure 6. Measured 235 U/ 238 U ratios of the reagents used in the dissolution, processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

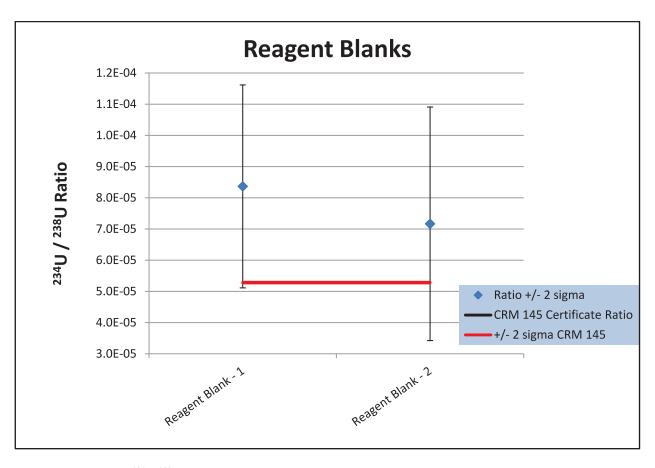


Figure 7. Measured ²³⁴U/²³⁸U ratios of reagents used in the dissolution, processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Swipes spiked with Standards

Figures 8 and 9 present the measured isotope ratios for $^{235}\text{U}/^{238}\text{U}$ and $^{234}\text{U}/^{238}\text{U}$ from the swipes spiked with the CRM 145 standard and processed with the described method. The $^{235}\text{U}/^{238}\text{U}$ ratio is at the natural abundance value. The $^{234}\text{U}/^{238}\text{U}$ ratio is enriched in ^{234}U , consistent with the results seen from the blank swipe. The expected $^{234}\text{U}/^{238}\text{U}$ ratio from a mixture of 2.4 ng of U with the average isotopic content of the blank swipes and 22 ng of CRM 145 would be 5.48 x 10^{-5} . The average of five measurements for this mixture was 5.45 x 10^{-5} , only 0.5% different from the expected. ^{236}U was not detected in these samples.

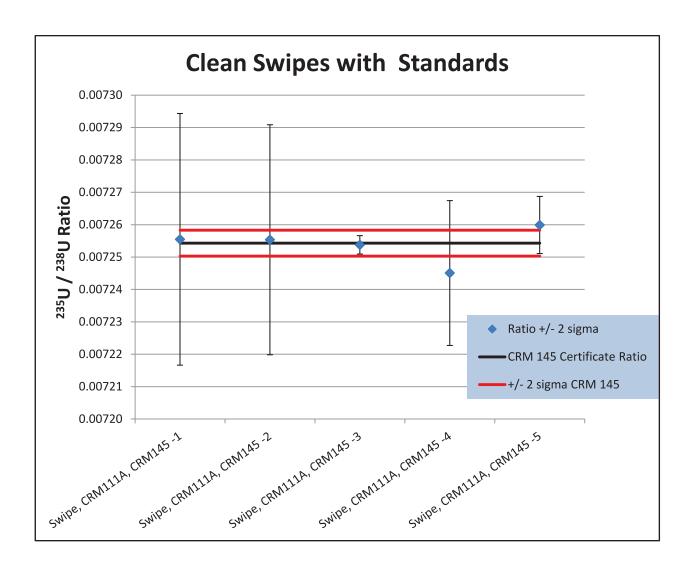


Figure 8. Measured 235 U/ 238 U ratios of clean swipes spiked with 22 ng of CRM 145 and processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.



Figure 9. Measured 234 U/ 238 U ratios of clean swipes spiked with 22 ng of CRM 145 and processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Figure 10 and 11 present the measured isotope ratios for $^{235}\text{U}/^{238}\text{U}$ and $^{234}\text{U}/^{238}\text{U}$ from the swipes spiked with the 4350B standard and processed with the described method. The measured $^{235}\text{U}/^{238}\text{U}$ ratio is at the natural abundance value of the standard. The measured $^{234}\text{U}/^{238}\text{U}$ ratio of 5.92 E-5 is enriched in ^{234}U compared to natural (5.46 E-5), but this is consistent with a published measurement of 5.82 E-5 for the $^{234}\text{U}/^{238}\text{U}$ ratio for this standard. This is a clear demonstration that the method can accurately detect a perturbation in the $^{234}\text{U}/^{238}\text{U}$ isotope ratio at a level of \sim 20 ng total uranium in the sample. ^{236}U was not detected in these samples.

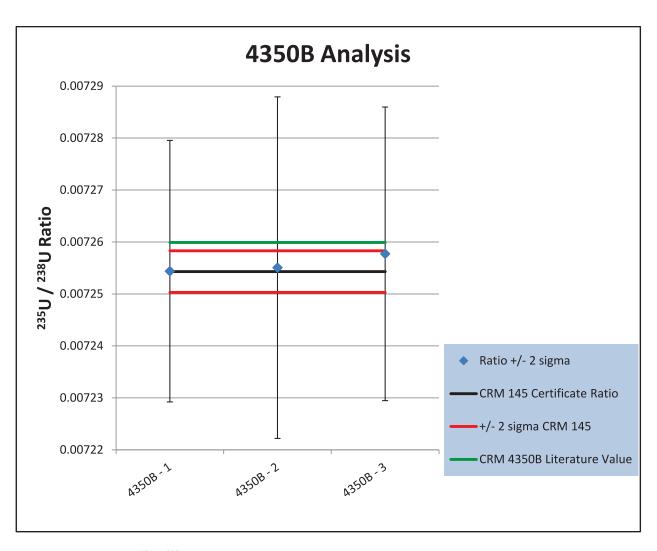


Figure 10. Measured ²³⁵U/²³⁸U ratios of clean swipes spiked with 4350B and processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

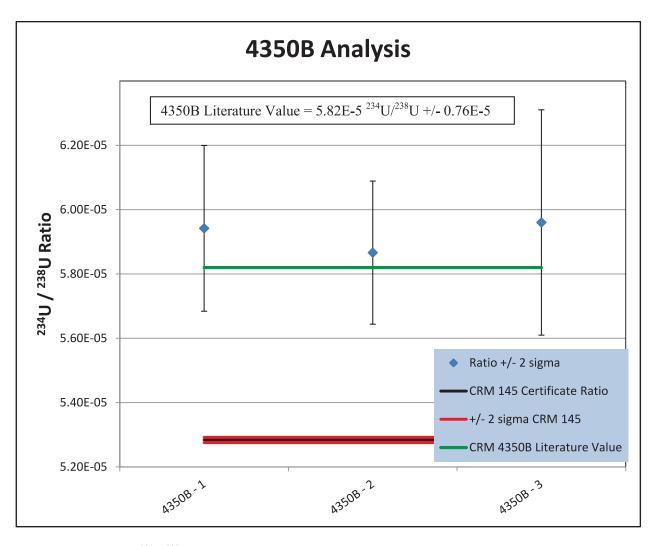


Figure 11. Measured ²³⁴U/²³⁸U ratios of clean swipes spiked with 4350B and processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Swipes of environmental samples

A variety of surfaces within the processing laboratory and the general INL environment were sampled. The first group was used to determine the background levels of U within the processing laboratory and its adjoining building, and the second group to demonstrate the viability of the methods on samples collected from an environment more typical of the anticipated application. Because of the long history of nuclear fuel development and processing at the INL it was anticipated that the isotope ratios would likely be enriched or depleted.

Figures 12 and 13 present the measured $^{235}\text{U}/^{238}\text{U}$ and $^{234}\text{U}/^{238}\text{U}$ ratios from the samples taken inside the clean room. The ratios are nearly identical to those measured for the blank swipe, the $^{235}\text{U}/^{238}\text{U}$ is at the natural level and the $^{234}\text{U}/^{238}\text{U}$ is enriched to the sample level detected for the blank swipe. Note that the total U present in these samples was only ~4 ng, consistent with the expected contribution from the blank swipe and

reagents. These results demonstrate that the clean rooms provide the necessary level of cleanliness.

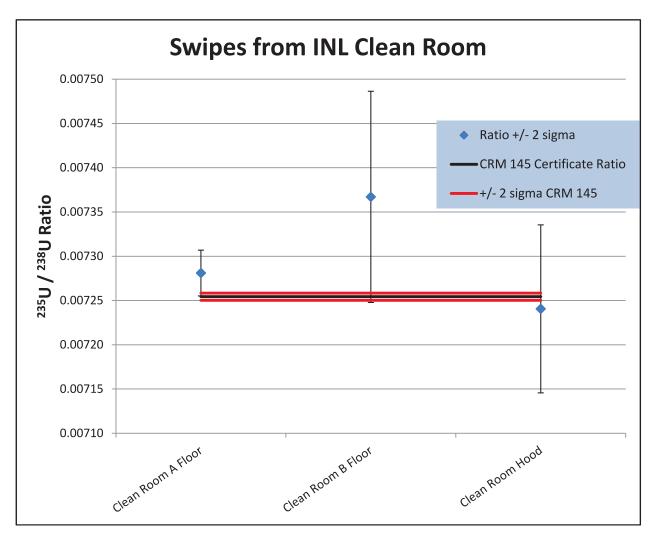


Figure 12. Measured ²³⁵U/²³⁸U ratios of swipes taken in the clean room processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

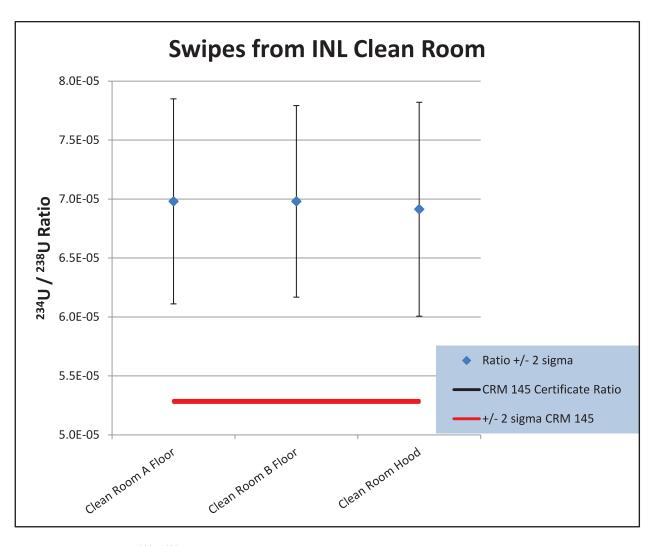


Figure 13. Measured ²³⁴U/²³⁸U ratios of swipes taken in the clean room processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Figures 14 and 15 present the measured ²³⁵U/²³⁸U and ²³⁴U/²³⁸U ratios for samples collected around the laboratory facility where the samples are processed. The samples were from two laboratories that were recently remodeled, from the common hallway for the building, from outside the building, and from the laboratory within the same building used to process gas standards. Generally the measured ²³⁵U/²³⁸U signatures were slightly depleted or natural. The sample from the 611-102 counter was the most depleted ²³⁵U/²³⁸U ratio observed and the ²³⁴U/²³⁸U ratio for this sample was the most enriched for samples from the building. This counter is in one of the remodeled laboratories. The craftsmen working on the remodel kept dedicated boots, and tools were cleaned as they entered the facility, but even with these measures it appears that some contamination was introduced. Note that no sample handling or processing takes place in these laboratories.

The last four data points show samples from inside the Gas Lab and associated office where operations have been conducted for a few years. The isotopic signatures are consistent with those seen in other parts of the building. There higher uncertainty in the

office and hood measurements is indicative of the small amount of uranium in the sample, likely due to the areas being kept cleaner. The final data point is from the surface of a gas sphere clam shell container which has been in many different off-site facilities. The total uranium for this sample (16 ng) is well above the blank level (2 ng); the 235 U/ 238 U ratio signature is natural and the 234 U/ 238 U ratio is slightly enriched, typical of most of the samples.

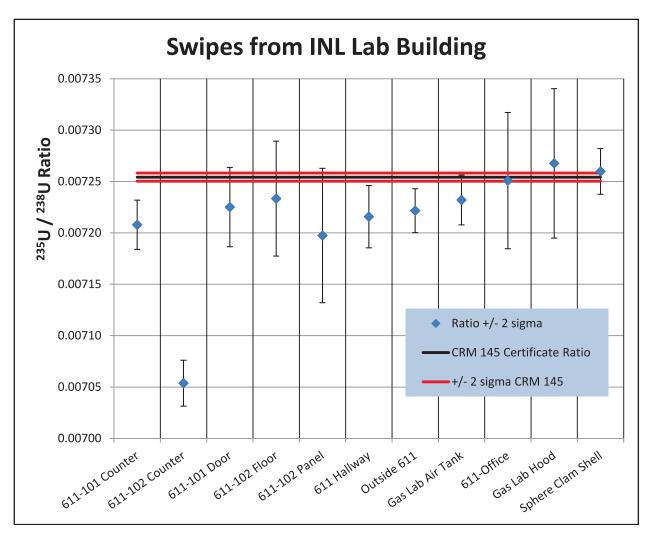


Figure 14. Measured 235 U/ 238 U ratios of swipes taken around INL building IF-611 processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show \pm -2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

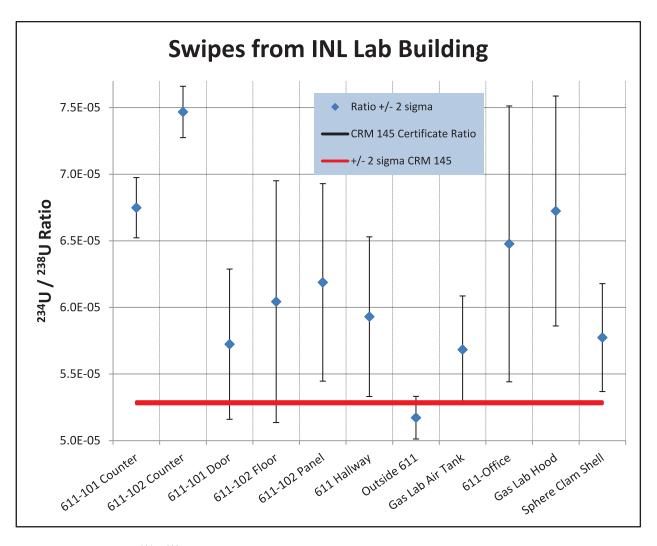


Figure 15. Measured ²³⁴U/²³⁸U ratios of swipes taken around INL building IF-611 processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Figures 16 and 17 present the measured $^{235}\text{U}/^{238}\text{U}$ and $^{234}\text{U}/^{238}\text{U}$ ratios for samples collected from a truck that travels to the INL Materials and Fuel Complex (MFC) and other INL locations on a daily basis, and from a xenon gas cylinder that had resided at MFC for many years and was recently moved to the IRC complex. The MFC is located \sim 50 miles west of the IRC. The sample from the truck has a natural $^{235}\text{U}/^{238}\text{U}$ ratio signature with a slightly enriched $^{234}\text{U}/^{238}\text{U}$ ratio, attributed to the contribution from the swipe.

Two samples were collected from the xenon gas cylinder, both showed an enriched ²³⁵U/²³⁸U signature and a ²³⁴U/²³⁸U signature typical of most samples. The enriched ²³⁵U/²³⁸U signature is reasonable given the legacy and ongoing fuel manufacturing, processing and examinations that occur at the MFC complex. These samples also included a signal at 236, which could be attributed to ²³⁶U as discussed below.

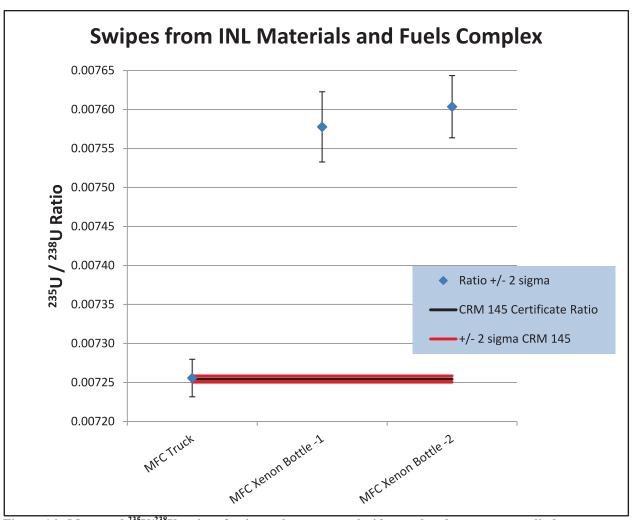


Figure 16. Measured ²³⁵U/²³⁸U ratios of swipes taken on a truck side panel and a xenon gas cylinder (twice), processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

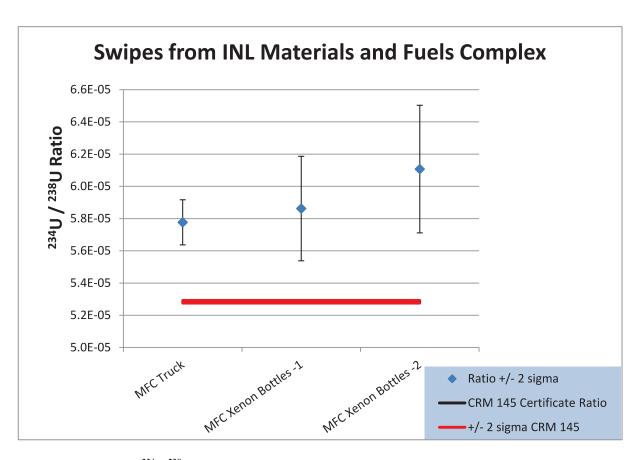


Figure 17. Measured ²³⁴U/²³⁸U ratios of swipes taken on a truck side panel and a xenon gas cylinder (twice), processed through the chemistry along with 3.3 pg of CRM 111A for quantification and measured by ICPMS. Data show +/- 2 sigma with the natural standard CRM 145 certified value shown on the bold black line and the uncertainty at 2 sigma is plotted in red.

Detection of 236U

After accounting for peak tailing from ²³⁵U and ²³⁸U the LOD for ²³⁶U was determined to be 9 million atoms (3.4 fg) and the LOQ 29 million atoms (11.4 fg). A signal above the LOD at mass 236 was detected on 6 of the samples. Two of these were at levels below the LOQ (611-101 counter and 611-101 door). The other four were found on swipes of the xenon gas bottle, the 611-102 counter, and the Clean Room A floor. The natural abundance of ²³⁶U is <1 x 10⁻¹¹ and thus is typically not detected in natural samples. However, it is a major component of uranium exposed to a neutron flux and thus a major component of reprocessed or spent nuclear fuel and thus becomes a signature for reactor operation or fuel reprocessing. Nuclear research and operations at the INL during its history make it highly likely that uranium samples from the main complexes (such as MFC) will contain ²³⁶U. The origins of the samples in this study in which a 236 signal was detected all have a link to the MFC complex. The xenon bottle spent years at the MFC and was not cleaned before being relocated to the IRC complex, thus it was not surprising to find a 236 signal in those two samples. The 611-101/2 counters and the 611-101 door were all in areas in which crafts personnel had recently spent considerable time remodeling. The crafts personnel perform tasks both at MFC and at the IRC, and their tools and equipment are used in both locations. While efforts were made to reduce the chance that contamination from MFC would be transferred to the IRC 611 building, it seems likely that this did occur. The clean room is located inside IRC 611 which has a

common hallway where personnel routinely move from one end of the hall to the next. This is the suspected route for the 236 contamination seen on the sample from Clean Room A floor. The clean rooms and all associated areas have since been cleaned but not yet re-sampled.

Data Summary

Table 1 presents all of the measured isotope ratios and the 2 sigma uncertainty for the samples discussed in this report. Where the minor isotope was below the LOD a ratio is not reported. Table 2 presents the quantity of each isotope measured above the LOD for all of the samples.

	²³⁴ U/ ²³⁸ U	²³⁴ U/ ²³⁸ U std	²³⁵ U/ ²³⁸ U	²³⁵ U/ ²³⁸ U std	²³⁶ U/ ²³⁸ U	²³⁶ U/ ²³⁸ U std
	•	dev of	•	dev of	Corrected	dev of
		measurement		measurement		measurement
Swipe, CRM111A, CRM 145 - 1	5.37E-05	2.19E-06	0.007255	0.000039	ND	-
Swipe, CRM111A, CRM 145 - 2	5.44E-05	1.81E-06	0.007255	0.000036	ND	-
Swipe, CRM111A, CRM 145 - 3	5.49E-05	3.25E-06	0.007254	0.000003	ND	-
Swipe, CRM111A, CRM 145 - 4	5.47E-05	2.31E-06	0.007245	0.000022	ND	-
Swipe, CRM111A, CRM 145 - 5	5.49E-05	1.98E-06	0.007260	0.000009	ND	-
CRM111A, CRM 145 - 1	5.29E-05	2.04E-06	0.007249	0.000022	ND	-
CRM111A, CRM 145 - 2	5.32E-05	2.81E-06	0.007250	0.000023	ND	-
CRM111A, CRM 145 - 3	5.27E-05	3.72E-06	0.007250	0.000027	ND	-
CRM111A, CRM 145 - 4	5.33E-05	3.45E-06	0.007252	0.000045	ND	-
CRM111A, CRM 145 - 5	5.29E-05	3.87E-06	0.007248	0.000038	ND	-
4350B - 1	5.94E-05	2.58E-06	0.007254	0.000025	ND	-
4350B - 2	5.87E-05	2.22E-06	0.007255	0.000033	ND	-
4350B - 3	5.96E-05	3.50E-06	0.007258	0.000028	ND	-
Swipe Blank - 1	7.13E-05	1.24E-05	0.007205	0.000129	ND	-
Swipe Blank - 2	7.44E-05	1.43E-05	0.007256	0.000112	ND	-
Swipe Blank - 3	7.20E-05	1.45E-05	0.007209	0.000064	ND	-
Reagent Blank - 1	8.37E-05	3.25E-05	0.007218	0.000595	ND	-
Reagent Blank - 2	7.17E-05	3.74E-05	0.007207	0.000314	ND	-
611-101 Counter	6.75E-05	2.26E-06	0.007208	0.000024	2.01E-07	7.6E-08
611-102 Counter	7.47E-05	1.93E-06	0.007054	0.000022	1.29E-06	2.7E-07
611-101 Back Door	5.72E-05	5.64E-06	0.007225	0.000039	2.40E-07	1.5E-07
611-102 Floor	6.04E-05	9.08E-06	0.007233	0.000056	ND	-
611-102 Electrical Panel	6.19E-05	7.41E-06	0.007197	0.000065	ND	-
611 - Hallway	5.93E-05	5.99E-06	0.007216	0.000030	ND	-
Outside 611	5.17E-05	1.60E-06	0.007222	0.000021	ND	-
Gas Lab Air Tank	5.68E-05	4.03E-06	0.007232	0.000024	ND	-
611-Office	6.48E-05	1.04E-05	0.007251	0.000066	ND	-
Gas Lab Hood	6.72E-05	8.63E-06	0.007268	0.000073	ND	-
Sphere Clam Shell	5.77E-05	4.05E-06	0.007260	0.000022	ND	-
MFC Truck	5.78E-05	1.40E-06	0.007256	0.000024	ND	-
MFC Xenon Bottles - 1	5.86E-05	3.24E-06	0.007578	0.000045	2.83E-06	3.4E-07
MFC Xenon Bottles - 2	6.11E-05	3.96E-06	0.007604	0.000040	6.88E-06	4.8E-07
Clean Room A Floor	6.98E-05	8.69E-06	0.007281	0.000026	2.25E-06	6.1E-07
Clean Room B Floor	7.74E-05	8.12E-06	0.007367	0.000119	ND	-
Clean Room Hood	6.91E-05	9.07E-06	0.007241	0.000095	ND	-

Table 1. Uranium isotopic ratios expressed with the standard deviation of the measurement (2-sigma).

	²³⁴ U (pg)	²³⁵ U (pg)	²³⁶ U (fg)	²³⁸ U (ng)
Swipe, CRM111A, CRM 145 - 1	1.3	177	ND	24
Swipe, CRM111A, CRM 145 - 2	1.3	175	ND	24
Swipe, CRM111A, CRM 145 - 3	1.3	176	ND	24
Swipe, CRM111A, CRM 145 - 4	1.3	176	ND	24
Swipe, CRM111A, CRM 145 - 5	1.3	176	ND	24
CRM111A, CRM 145 - 1	1.2	161	ND	22
CRM111A, CRM 145 - 2	1.2	159	ND	22
CRM111A, CRM 145 - 3	1.2	160	ND	22
CRM111A, CRM 145 - 4	1.2	162	ND	22
CRM111A, CRM 145 - 5	1.2	161	ND	22
4350B - 1	2.2	271	ND	37
4350B - 2	2.3	283	ND	39
4350B - 3	2.4	289	ND	40
Swipe Blank - 1	0.17	17	ND	2.4
Swipe Blank - 2	0.16	15	ND	2.1
Swipe Blank - 3	0.17	17	ND	2.3
Reagent Blank - 1	0.02	1.9	ND	0.3
Reagent Blank - 2	0.02	2.2	ND	0.3
611-101 Counter	3.6	388	10.8	54
611-102 Counter	1.8	174	31.8	25
611-101 Back Door	1.2	147	4.9	20
611-102 Floor	0.53	64	ND	8.8
611-102 Electrical Panel	0.37	43	ND	5.9
611 - Hallway	0.80	98	ND	14
Outside 611	5.9	825	ND	114
Gas Lab Air Tank	1.0	133	ND	18
611-Office	0.34	38	ND	5.2
Gas Lab Hood	0.38	41	ND	5.6
Sphere Clam Shell	0.95	119	ND	16
MFC Truck	1.9	241	ND	33
MFC Xenon Bottles - 1	1.2	154	57	20
MFC Xenon Bottles - 2	1.2	149	134	20
Clean Room A Floor	0.43	45	13.8	6.1
Clean Room B Floor	0.19	18	ND	2.4
Clean Room Hood	0.28	29	ND	4.0

Table 2. Mass content by isotope for uranium.

Statistical test of difference

To systematically determine if a measured isotope ratio differs from the natural ratio a test (of hypothesis) for a significant difference between a measured value and the

certified standard value was applied as follows. The certified 235 U/ 238 U atom ratio for CRM-145 is 0.0072543, with an uncertainty at the 95% confidence level (two standard deviations) of 0.000004. Multiple measurements of CRM-145 on the ICPMS result in an estimated processed sample uncertainty (two standard deviations) of $2\hat{\sigma}$. With approximate 95% confidence, a measured 235 U/ 238 U atom ratio, denoted by Y, will significantly differ from the certified 235 U/ 238 U atom ratio (thereby indicating non-natural uranium in the sample), if

Equation 3.
$$|Y - 0.0072543| > 2\sqrt{\hat{\sigma}^2 + 0.000002^2}$$

where

$$Y = {}^{235}U/{}^{238}U$$
 measured

σ = the processed sample uncertainty determined by using the standard deviation of the results from the most recent 20 blank filters spiked with CRM 145 at 20 ng total uranium.

Applying this test the ratio perturbation detectable by this method is \pm 1 x 10⁻⁵ (0.007254 \pm 7 - 0.000010) from natural for \pm 235U / \pm 238U. To date ten measurements have been collected and these were used to generate the sigma in applying this test to the CRM 4350B results presented in Table 1. In the future it is proposed that this test would be applied using a 20 sample moving average of $\hat{\sigma}$, the estimated processed sample uncertainty obtained via measurement of CRM-145 on the ICPMs. The 20 sample moving average will be implemented once the data set is larger than 20. This same method was applied to the \pm 234U/238U ratios of CRM 4350B shown in Table 1. This test shows samples that would be considered to have natural isotopic content. Table 3 shows that the results from this test confirm the discussion above. The \pm 235U / 238U ratio is natural in 4350B and the \pm 234U / 238U ratio is not natural. The literature values reported for 4350B are in agreement with these findings.

Sample	Natural	Natural	
	²³⁴ U/ ²³⁸ U	²³⁵ U/ ²³⁸ U	
4350B - 1	NO	YES	
4350B - 2	NO	YES	
4350B - 3	NO	YES	

Table 3. Application of the test of difference using the Columbia River Sediment standard 4350B, which has a natural 235 U / 238 U ratio and a non-natural 234 U / 238 U ratio.

Conclusions and Discussion

Advances in Inductively Coupled Plasma Mass Spectrometry over the last decade have enabled analyses of U and Pu that rival that of TIMS. This has been coupled with a simplified chemical extraction and separation process to provide the capability to quantitatively measure uranium on swipes faster and at less cost than previous methods. The measurements reported herein have clearly demonstrated that this capability provides the necessary level of detection and specificity for the uranium isotopes of interest. The minimum detectable amount of 236 U (LOD, defined as 3σ) for 236 U is 9 million atoms (3.4 fg 236 U). The limit of quantification (LOQ defined as $10~\sigma$) for 236 U is 29 million atoms (11.4 fg 236 U). The measured limit of detection for 238 U from the CRM 145 spiked onto a swipe is 7.1 x 10^{11} atoms of 238 U (0.28 ng) with the limit of quantification being 2.4 x 10^{12} atoms of 238 U (0.93 ng). The ratio perturbation for 235 U / 238 U detectable by this method is +/- 1 x 10^{-5} (0.007254 +/- 0.000010) from natural. Analysis of the environmental standard 4305B (Columbia River Sediment) demonstrated the capability of the method to accurately measure the enriched 234 U/ 238 U ratio from a sample containing only 40 ng of uranium.

The INL is confident that it is capable of quickly and efficiently detecting trace levels of uranium and plutonium present on swipe materials using methods and instrumentation that greatly reduce the cost and time of analysis. These new methods permit larger batch processing and analysis, produce less waste, and enable automated sample analysis on the mass spectrometer. In addition, for a time critical sample the new methods make it possible to turn around a sample from receipt to report of results the following day.